

Point Lepreau Nuclear Generating Station PO Box 600, Lepreau, NB E5J 286

TU 06374

September, 21, 2018

Mr. Brian Torrie, Director General Regulatory Policy Directorate Canadian Nuclear Safety Commission 280 Slater Street P.O. Box 1046, Station B Ottawa, Ontario K1P 5S9

Dear Mr. Torrie:

Subject: NB Power Comments on REGDOC 1.1.5 - Licence Application Guide: Small Modular Reactor Facilities

The purpose of this letter is to provide NB Power's comments on REGDOC 1.1.5 -Licence Application Guide: Small Modular Reactor Facilities (Reference 1).

NB Power's Point Lepreau Nuclear Generating Station (PLNGS) has collaborated with industry to review the proposed regulatory document in detail.

PLNGS appreciates the opportunity to provide input to strengthen the licencing process. Please see Attachment 1 for a full list of comments and clarification requests for the CNSC to consider.

NB Power is prepared to clarify our comments and concerns. If you require additional information, please contact Brian Thorne at 506-659-6264 or brthorne@nbpower.com.

Sincerely,

**Brett Plummer** 

Vice President Nuclear and Chief Nuclear Officer

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BP/BT/bt

cc. Bruno Romanelli, Isabelle Gingras, Josée Giguère, Nathan Kline, Carleigh Zelmer (CNSC - Ottawa)
consultation@cnsc-ccsn.gc.ca
CNSC Site Office
Carol Murray, Amanda Gardner, Scott Demmons, Krista Ward, Brian Thorne (NBP)

### Reference:

1. CNSC draft REGDOC 1.1.5, Licence Application Guide: Small Modular Reactor Facilities, July 2018

### Attachment:

#	Document/	Industry Issue	Suggested Change (if applicable)	Major	Impact on Industry, if major comment
	Excerpt of Section			Comment/ Request for Clarification 1	
1.	General	The term Small Modular Reactor (SMR) has not been well defined within a regulatory framework.  REGDOC-3.6, Glossary of CNSC Terminology does define the term "small reactor" as referenced on page 24 of REGDOC-1.1.5. However, there is no definition for "Small Modular Reactor" in REGDOC 3.6, which indicates that "SMR" stands for "scheduled maintenance release" or "security monitoring room."  While SMR generally means reactors that are smaller in size than current generation baseload nuclear power plants, SMRs encompass a broad range of reactor technology from conventional to highly-innovative. It would be useful to both licensees and regulators to establish a technical descriptor for these types of reactors with novel technologies.	Industry suggests the CNSC should consider adopting the IAEA definition of SMR, which reads: "Small modular reactors are defined as advanced reactors that produce electricity of up to 300 MW(e) per module. These reactors have advanced engineered features, are deployable either as a single or multi-module plant, and are designed to be built in factories and shipped to utilities for installation as demand arises."	MAJOR	A technical descriptor for these types of reactors with novel technologies would help the CNSC and industry reach a common understanding of what is meant by SMR.  Without a clear definition, it is more difficult to understand how the application of the guidance in REGDOC 1.1.5 would differ from conventional nuclear power plants and small reactors (as defined in RD-367, Design of Small Reactor Facilities).
2.	General	Terminology is used inconsistently in this draft REGDOC. For example:  The second pre-licensing activity is referred to as "pre-licensing engagement" in Section 1.2 and as "the process for establishing an appropriate strategy for risk-informed licensing" in Appendix B.  The document uses terms such as "proponent" and "applicant" or "applicant/licensee in several	Use the same terminology throughout the document to avoid confusion.	Clarification	

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"	Excerpt of Section			Comment/	
	Executed of Grands			Request for	
				Clarification 1	
		sections such 1.2, B.2.1 and Appendix			
1		B. Are they to be treated as			
1		synonymous?			
		In Section A.2, "management			
		systems" and "quality assurance" are			
		redundant. Please remove the term			
		"quality assurance."			
		Some sections, such as 2.3, use the	'		
		term "Aboriginal" while others such			
		as A.1 and B.2.1 use "Indigenous."			
į.		Licensees suggest Indigenous be used			
		consistently throughout.			
+		Replace the phrase "to build" with			
		"to construct" in the 1st paragraph of			
		Section 1.2 for consistent use of			
		terminology.			
3.	General	Access to Information requirements are	In future drafts, industry suggests	Clarification	
		not addressed in this initial draft.	providing contextual information		
			on the Access to Information Act		
			regarding pre-licensing submission		
			applicability and opportunities for		
			applicants to protect/remove		
0			sensitive information. This would		
			be most beneficial to new		
			applicants.		
4.	1.1	How does this guideline differ from a		Clarification	
Ì		research reactor guide? If a low-powered			
		SMR is not connected to a power			
		conversion unit, can it be licensed as a			
		research reactor facility? Also, it is not			
		clear what "alternative approaches"			
		refers to in the 4 <sup>th</sup> paragraph of this			
		section.			

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				Request for	
-	1.1	This document is intended to be used in	For clarity, industry suggest future	Clarification <sup>1</sup> Clarification	
5.	1.1	consultation with CNSC staff and the	drafts of this document refer to	Clarification	
		following regulatory documents, which	the full suite of REGDOCS		
		detail requirements and guidance for an	applicable to SMRS, possibly		
		applicant to review prior to submitting a	displayed using a ball-and-stick		
		licence application:	diagram.		
		REGDOC-1.1.1, Licence to Prepare			
		Site and Site Evaluation for New Reactor Facilities	1		
		RD/GD-369, Licence Application			
		Guide: Licence to Construct a Nuclear			
		Power Plant			
		REGDOC-1.1.3, Licence Application			
		Guide: Licence to Operate a Nuclear			
		Power Plant			
		However, REGDOC-1.1.5 does not			
		provide a straightforward alignment			
		between those three REGDOCs and the			
		CNSC's 14 Safety and Control Areas,			
		especially for those who are not fully knowledgeable about the Nuclear Safety			
		and Control Act and Regulations and the			
		licence application requirements.			
		Also, as currently written, REGDOC-1.1.5			
		does not provide guidance to licence			
		applicants for SMR facilities regarding the extent of information and			
		completeness of their facility design that			
		is necessary to apply for licences to			
		prepare a site, construct a facility and			
		operate it.			

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	Enter pe er estation			Request for Clarification 1	
6.	2.2.1	Clarity is sought regarding item #8 and the reference to barriers. Barriers against what? Release of radioactive materials to the environment?	Clarify what is meant by barriers.	Clarification	
7.	2.2.2	Ensure the SCAs being exempted from the application to prepare a site are consistent with other documents, such as <i>REGDOC- 1.1.1</i> , and the REGDOCs cited on the CNSC website.	Ensure consistency of exemptions in all CNSC material.	Clarification	
8.	2.2.5	The third paragraph incorrectly references the "management system SCA" rather than the "physical design SCA."	Amend the sentence to read, "Consult the CNSC's Regulatory documents Web page for a list of regulatory documents and CSA standards that may be applicable to the management system physical design SCA"	Clarification	
9.	2.2.10	To be consistent with the topic of this SCA, the note in the 2 <sup>nd</sup> paragraph should include the phrase "fire protection."	Amend the 2 <sup>nd</sup> paragraph to read,  "Note: The emergency management and fire protection SCA includes "	Clarification	
10	2.2.14	Bullet #2 simply repeats the parent statement. If the bullet is meant to capture the heading for Section 4.14.2 in REGDOC-1.1.3, Licence Application Guide: Licence to Operate a Nuclear Power Plant it should be:  "2. Packaging and transport program."	Suggest changing to: "2. Packaging and transport <u>program</u> ."	Clarification	
11	2.3	There is duplication between Section 2.2.11 and Section 2.3 on financial guarantees.	Financial guarantees are already covered in Section 2.3 and should be removed from Section 2.2.11.	Clarification	
12	B.1	It is redundant to use the word "optional" twice in the last sentence of the 1st paragraph.	Amend the sentence to read: "The CNSC offers an optional vendor design review (VDR)	Clarification	

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77	Excerpt of Section	Triviolity 10040	and the second	Comment/	
	Except of obtain			Request for	
				Clarification 1	
			optional service in this regard.		
13	B.2	The final sentence in the 2 <sup>nd</sup> paragraph of	Amend the 2nd paragraph to read:	Clarification	
		Section B.2 repeats the meaning of a	" the approach to operation.		
		statement already included at the end of	Information acquired through a		
		Section B.1.	VDR can be very useful, and can		
			be used in the licensing process at	10	
			the applicant's discretion."	, , , , , , , , , , , , , , , , , , ,	V
14.	Preface; 1, 1.1, Appendix A	In the stated sections, this draft refers to	Clarify whether this guide is also	Clarification	
		licence to prepare site, licence to	applicable to licence to		
		construct and licence to operate. Is this	decommission and licence to	8	
		guide also applicable to licence to	abandon and add those		
		decommission and licence to abandon?	references if necessary.		
15.	1.1	In the discussion in Appendix B and VDR/	Ensure both pre-licensing	Clarification	
	Last paragraph;	pre-licensing, VDR is identified as	processes are clearly identified as		
	Appendix B, Figure 1	optional. However, no similar statement	optional in future drafts.		
	1.2 2 <sup>nd</sup> bullet	is included in relation to pre-licensing engagement activities by potential		İ	
	Appendix B	licensees.			
	Page 17	neerisees.			
16	2.1	The bullets in the 1 <sup>st</sup> paragraph refer to	Ensure all applicable licence and	Clarification	
		specific sections of the General Nuclear	regulation references are clearly	8	
		Safety and Control Regulations, Class I	identified.		
		Nuclear Facilities Regulations and Nuclear	1		
		Security Regulations, paragraph 3 (b). The			
		regulations reference seems specific to a			
		Licence to Prepare Site and is not		1	
		comprehensive to include other types of			
		licences. For example, Class I Section 5 (a)			
		to (m) would relate to an application for			
		a Licence to Construct and Class I Section		1	
		6 (a) to (n) would be applicable for a Licence to Operate. Similarly, other			
l		subsections of General Nuclear Safety			
	<u> </u>	Januacettonia of General Muclear Safety	l		D F/0

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		and Control Regulations would be applicable.			
17	2.1	With regard to subsections 1-3 in the 2 <sup>nd</sup> paragraph, the list of general information to satisfy the regulations for all types of licenses is not comprehensive. For example, information on environmental releases, waste, malfunction, accidents etc. is not mentioned.	Industry suggests future drafts of this document refer to the appropriate REGDOCs.		
18	2.2.2	Human Factors is not included in the list even though <i>REGDOC 2.2.1</i> is mentioned on the CNSC Regulatory Documents web page under the NSCA.	Industry seeks clarity on whether Human Factors was purposefully omitted for this REGDOC.	Clarification	
19	2.2.4	It is anticipated that compliance with CSA standards could be a concern for offshore SMR vendors who do not use current CSA standards.	Industry suggests the CNSC consider providing additional context and guidance in this section on how to address this issue.	Clarification	
20	2.2.5	Many of the considerations in the 2 <sup>nd</sup> list in this section may be unknown during the pre-licensing engagement phase as well as at LTPS since final design details or even intended technology may not have been finalized.	Industry suggests future drafts of this REGDOC acknowledge this potential gap.	Clarification	
21	2.2.10	The scope of agencies involved is incomplete in bullet #4 in the 5 <sup>th</sup> paragraph.	Industry suggests amending the bullet to read, "Ability of the applicant, region/municipality and/or province/territory to respond to an emergency"	Clarification	
22.	2.2.10	Is exclusion of "Drills & Exercises" an omission or by intent from the list of considerations the applicant should address when assessing the emergency	Clarify whether the omission was intentional.	Clarification	

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		and management and fire protection SCA?		Ciarification	
23	2.3	Beneath the 1 <sup>st</sup> paragraph, "Intergovernmental consultation" is listed as one of the four other regulatory areas. The 2 <sup>nd</sup> paragraph then says, "Consult the CNSC's Regulatory documents Web page for the above- mentioned four matters." It seems this topic is not explicitly mentioned on the CNSC's website.	Industry suggests future drafts of the REGDOC be revised to include the need to consult the CNSC on "Intergovernmental consultation."	Clarification	
24	Appendix A	The last four paragraphs on Page 14 are well-written and high value since they encompass the essence of licensing an SMR, i.e. the use of a risk-informed, graded approach and the consideration of professional judgement. This information needs to be stated up front in the document, not relegated to an Appendix.	Industry suggests the CNSC move or copy the identified text to the Preface/Introduction in future drafts of this REGDOC.	Clarification	
25.	Appendix B	Figure 1 is incomplete.	Phase 3 VDR should be included in Figure 1 and "Activity D" should be cross-referenced to Section B.2.1	Clarification	
26.	B.3	The 1 <sup>st</sup> paragraph contains the phrase, "to initiate a technical assessment." Is the applicant expected to provide any additional information to support the "technical assessment"?	If additional information is required, please specify.	Clarification	
27.	B.3.1,	The 4 <sup>th</sup> bullet under 'Description of the nuclear facility or activity' says, "The proposed organizational arrangements for the conduct of the activities to be	Eliminate the repetition in future drafts.	Clarification	

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		licensed." This essentially repeats the final bullet on the list under 'Description of the purpose of the project and key activities to be conducted,' which says, "key project activities and organizational arrangements that have been identified for each project phase."			
28	B.3.1	Bullet # 5 under 'Description of the nuclear facility or activity' is in the wrong place.	Industry suggests this bullet should be moved to Appendix B, Section B.3.1, Page 23 under 'Estimate of quantity, form, origin and volume of any radioactive waste or hazardous waste.'	Clarification	